

KANSAS CITY POWER & LIGHT COMPANY

1330 BALTIMORE AVENUE

P.O. BOX 679

KANSAS CITY, MISSOURI 64141

LAW DEPARTMENT
(816) 556-2785

November 14, 1986

File No. 0502-50

NOV 17 1986
RECEIVED

Mr. Harvey G. Hubbs
Secretary
Missouri Public Service Commission
P. O. Box 360
Jefferson City, Missouri 65102

Re: Case No. EF-87-29;
Kansas City Power & Light Company
Supplemental Exhibit to Application

Dear Mr. Hubbs:

On October 1, 1986, KCPL filed in this case an Application for Authority to Acquire Certain Shares of Stock of Wolf Creek Nuclear Operating Corporation (WCNOC). Paragraph 8(b) of the Application stated that an application was filed with the Nuclear Regulatory Commission (NRC), seeking an amendment to the Wolf Creek Facility Operating License to permit WCNOC to assume the operating responsibilities held thereunder by Kansas Gas and Electric Company.

On November 4, 1986, the NRC issued Amendment No. 4 to the Facility Operating License, permitting the licensed activities to be under the control of WCNOC. The amendment is effective January 1, 1987.

KCPL hereby supplements its Application by tendering fifteen copies of said Amendment No. 4 and accompanying cover letter as Exhibit F to its Application, so that the Commission may review this NRC approval while considering KCPL's Application.

Please bring this to the Commission's attention.

Mark G. English
Mark G. English

MGE:cg

Enc.

cc: Steven Dottheim
Douglas Brooks
A. Drue Jennings

FILED

NOV 17 1986

PUBLIC SERVICE COMMISSION



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

November 4, 1986

GLENN L. KOESTER
RECEIVED

Docket No.: 50-482

NOV 10 1986

Mr. Glenn L. Koester
Vice President - Nuclear
Kansas Gas and Electric Company
201 North Market Street
Post Office Box 208
Wichita, Kansas 67201

Dear Mr. Koester:

Subject: Wolf Creek Generating Station - Amendment No.4 to Facility
Operating License NPF-42

The Commission has issued the enclosed Amendment No. 4 to Facility Operating License NPF-42 for the Wolf Creek Generating Station. The amendment is in response to your application dated April 15, 1986, and supplemented July 29, 1986.

This amendment modifies the operating license and Technical Specifications to permit licensed activities to be under the control of the Wolf Creek Nuclear Operating Corporation. The amendment is effective January 1, 1987.

A copy of our related Safety Evaluation is enclosed. Notice of issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Paul W. O'Connor".

Paul W. O'Connor, Project Manager
PWR Project Directorate #4
Division of PWR Licensing-A

Enclosures:

1. Amendment No. 4 to License No. NPF-42
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. Glenn L. Koester
Kansas Gas and Electric Company

Wolf Creek Generating Station
Unit No. 1

CC:

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Executive Director, SNUPPS
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

KANSAS GAS & ELECTRIC COMPANY
KANSAS CITY POWER AND LIGHT COMPANY
KANSAS ELECTRIC POWER COOPERATIVE, INC.
WOLF CREEK GENERATING STATION
DOCKET NO. 50-482
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 4
License No. NPF-42

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Wolf Creek Generating Station (the facility) Facility Operating License No. NPF-42 filed by Kansas Gas and Electric Company acting for itself and Kansas City Power and Light Company and Kansas Electric Power Cooperative, Inc., (the licensee) dated April 15, 1986, and supplemented July 29, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, Facility Operating License No. NPF-42 is changed as follows:
 - A. Change paragraph 1.E. to read as follows:

Wolf Creek Nuclear Operating Corporation* is technically qualified to engage in the activities authorized by this license in accordance with the Commission's regulations set forth in 10 CFR Chapter 1;

- B. Change the footnote related to paragraph 1.E. to read as follows:

*Wolf Creek Nuclear Operating Corporation is authorized to act as agent for the Kansas Gas and Electric Company, Kansas City Power & Light Company, and Kansas Electric Power Cooperative, Inc., and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

- C. Change paragraph 2.A. to read as follows:

The license applies to the Wolf Creek Generating Station, Unit No. 1, a pressurized water nuclear reactor and associated equipment (the facility), owned by Kansas Gas and Electric Company, Kansas City Power & Light Company, and Kansas Electric Power Cooperative, Inc., and operated by the Wolf Creek Nuclear Operating Corporation. The facility is located in Coffey County, Kansas, approximately 28 miles east-southeast of Emporia, Kansas, and is described in the licensees' "Final Safety Analysis Report," as supplemented and amended, and in the licensees' Environmental Report, as supplemented and amended.

- D. Change paragraph 2.B. to read as follows:

Subject to the conditions and requirements incorporated herein, the Commission hereby licenses Wolf Creek Nuclear Operating Corporation (the Operating Corporation), Kansas Gas and Electric Company (KG&E), Kansas City Power & Light Company (KCPL), and Kansas Electric Power Cooperative, Inc. (KEPCO):

- E. Change paragraph 2.B.(1) to read as follows:

Pursuant to Section 103 of the Act and 10 CFR Part 50 "Domestic Licensing of Production and Utilization Facilities," the Operating Corporation, to possess, use and operate the facility at the designated location in Coffey County, Kansas, in accordance with the procedures and limitations set forth in this license;

- F. Change paragraph 2.B.(2) to read as follows:

KG&E, KCPL and KEPCO to possess the facility at the designated location in Coffey County, Kansas, in accordance with the procedures and limitations set forth in this license;

- G. Change paragraph 2.B.(3) to read as follows:

The Operating Corporation, pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;

H. Change paragraph 2.B.(4) to read as follows:

The Operating Corporation, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

I. Change paragraph 2.B.(5) to read as follows:

The Operating Corporation, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any by-product, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

J. Change paragraph 2.B.(6) to read as follows:

The Operating Corporation, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

K. Change paragraph 2.C.(1) to read as follows:

The Operating Corporation is authorized to operate the facility at reactor core power levels not in excess of 3411 megawatts thermal (100% power) in accordance with the conditions specified herein and in Attachment 1 to this license. The activities identified in Attachment 1 to this license shall be completed as specified. Attachment 1 is hereby incorporated into this license.

L. Change paragraph 2.C.(2) to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 4, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into the license. The Operating Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

M. Change paragraph 2.C.(5) to read as follows:

Fire Protection (Section 9.5.1, SER, Section 9.5.1.8, SSER #5)

- (a) The Operating Corporation shall maintain in effect all provisions of the approved fire protection program as described in the SNUPPS Final Safety Analysis Report for the facility through Revision 17, the Wolf Creek site addendum through Revision 15, and as approved in the SER through Supplement 5, subject to provisions b & c below.

- (b) The Operating Corporation may make no change to the approved fire protection program which significantly would decrease the level of fire protection in the plant without prior approval of the Commission. To make such a change the licensee must submit an application for license amendment pursuant to 10 CFR 50.90.
- (c) The Operating Corporation may make changes to features of the approved fire protection program which do not significantly decrease the level of fire protection without prior Commission approval, provided:
 - (i) such changes do not otherwise involve a change in a license condition or technical specification or result in an unreviewed safety question (see 10 CFR 50.59).
 - (ii) such changes do not result in failure to complete the fire protection program approved by the Commission prior to license issuance.

The Operating Corporation shall maintain, in an auditable form, a current record of all such changes including an analysis of the effects of the change on the fire protection program and shall make such records available to NRC inspectors upon request. All changes to the approved program shall be reported to the Director of the Office of Nuclear Reactor Regulation, along with the FSAR revisions required by 10 CFR 50.71(e).

- N. Change paragraph 2.C.(6) to read as follows:

Qualification of Personnel (Section 13.1.2, SSER #5, Section 18, SSER #1)

The Operating Corporation shall have on each shift operators who meet the requirements described in Attachment 2.

- O. Change paragraph 2.C.(7) to read as follows:

NUREG-0737 Supplement 1 Conditions (Section 22, SER)

The Operating Corporation shall complete the requirements described in Attachment 3 to the satisfaction of the NRC. These conditions reference the appropriate items in Section 22, "TMI Action Plan Requirements for Applicants for Operating Licenses," in the Safety Evaluation Report and Supplements 1, 2, 3, 4, and 5 of NUREG-0881.

- P. Change paragraph 2.C.(11) to read as follows:

Steam Generator Tube Rupture (Section 15.4.4, SSER #5)

Prior to restart following the first refueling outage, the Operating Corporation shall submit for NRC review and approval an analysis which demonstrates that the steam generator single-tube rupture (SGTR) analysis presented in the FSAR is the most severe case with respect to the release of fission products and calculated doses. Consistent with the analytical assumptions, the licensee shall propose all necessary changes to Appendix A to this license.

- Q. Change paragraph 2.C.(12) to read as follows:

LOCA Reanalysis (Section 15.3.7, SSER #5)

Prior to restart following the first refueling outage, the Operating Corporation shall submit for NRC review and approval a reanalysis for the worst large break LOCA using an approved ECCS evaluation model.

- R. Change paragraph 2.C.(13) to read as follows:

Generic Letter 83-28

The Operating Corporation shall submit responses to and implement the requirements of Generic Letter 83-28 on a schedule which is consistent with that given in their February 29, 1984 and February 6, 1985 letters.

- S. Change paragraph 2.C.(14) to read as follows:

Surveillance of Hafnium Control Rods (Section 4.2.3.1(10), SER and SSER #2)

The Operating Corporation shall perform a visual inspection of a sample of hafnium control rods during one of the first five refueling outages. A summary of the results of these inspections shall be submitted to the NRC.

- T. Change paragraph 2.E. to read as follows:

The Operating Corporation shall fully implement and maintain in effect all provisions of the Commission approved Physical Security, Guard Training and Qualification, and Safeguards Contingency plans, including all amendments and revisions made pursuant to the authority of 10 CFR 50.90 and 10 CFR 50.54(p), which are part of the license. These plans, which contain Safeguards Information protected under 10 CFR 73.21 are entitled "Wolf Creek Generating Station, Physical Security Plan; Safeguards Contingency Plan and the Security Training and Qualification Plan".

- U. Change Attachment 2 to read as follows:

The Operating Corporation shall have a licensed senior operator on each shift who has had at least six months of hot operating experience on a same type plant, including at least six weeks at power levels greater than 20% of full power, and who has had startup and shutdown experience. For those shifts where such an individual is not available on the plant staff, an advisor shall be provided who has had at least four years of power plant experience, including two years of nuclear plant experience, and who has had at least four years of power plant experience, including two years of nuclear plant experience, and who has at least one year of experience on shift as a licensed senior operator at a similar type facility. Use of advisors who were licensed only at the RO level will be evaluated on a case-by-case basis. Advisors shall be trained on plant procedures, technical specifications and plant systems, and shall

be examined on these topics at a level sufficient to assure familiarity in the role of the advisors. These advisors, or fully trained and qualified replacements, shall be retained until the experience levels identified in the first sentence above have been achieved. The names of any replacement advisors shall be certified by the Operating Corporation prior to these individuals being placed on shift. The NRC shall be notified at least 30 days prior to the date the Operating Corporation proposes to release the advisors from further service.

V. Change Attachment 3 to read as follows:

NUREG-0737, SUPPLEMENT 1, REQUIREMENTS

(1) Functional and Task Analysis (I.C.1, SSER #5)

Prior to startup following the first refueling outage, the Operating Corporation shall submit for staff review and approval, a description of the process used to complete the functional and task analysis, including a description and justification for all information and control deviations from the Westinghouse Owners Group Emergency Response Guidelines, Revision 1.

(2) Emergency Response Capabilities (Generic Letter 82-33, Supplement 1 to NUREG-0737)

Prior to restart following the first refueling outage, the Operating Corporation shall have a fully functional Technical Support Center and Emergency Operations Facility and a fully operable Emergency Response Facilities Information System (ERFIS).

(3) Regulatory Guide 1.97 (Section 7.5.2.3, SSER #3)

Prior to restart following the first refueling outage, the Operating Corporation shall have installed and operable the following instrumentation.

- (a) Source range instrumentation qualified to post-accident conditions
- (b) Reactor vessel water level instrumentation
- (c) Subcooling monitors
- (d) Radiation monitors for releases from steam generator safety/relief valves or atmospheric dump valves, and
- (e) Auxiliary feedwater pump turbine exhaust monitor

3. Facility Operating License No. NPF-42 is further amended by changes to the Technical Specifications as indicated in the attachment to this license amendment.
4. This license amendment is effective January 1, 1987.

FOR THE NUCLEAR REGULATORY COMMISSION

Paul W. O'Connor

Paul O'Connor, Project Manager
PWR Project Directorate #4
Division of PWR Licensing-A

Attachment:
Changes to the
Technical Specifications

Date of issuance: November 4, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 4

OPERATING LICENSE NO. NPF-42

DOCKET NO. 50-482

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

AMENDED PAGE

6-1
6-3
6-8
6-9
6-10
6-12
6-13
6-14
6-15

OVERLEAF PAGE

6-2
6-4
6-7

6-11

6-16

ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Plant Manager shall be responsible for overall Unit operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Supervising Operator, under the Shift Supervisor, shall be responsible for the control room command function. A management directive to this effect, signed by the President and Chief Executive Officer shall be reissued to all station personnel on an annual basis.

6.2 ORGANIZATION

OFFSITE

6.2.1 The Operating Corporation organization for unit management and technical support shall be as shown in Figure 6.2-1.

UNIT STAFF

6.2.2 The Unit organization shall be as shown in Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1;
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor. In addition, while the Unit is in MODE 1, 2, 3 or 4, at least one licensed Senior Operator shall be in the control room;
- c. An individual from the Health Physics Group*, qualified in radiation protection procedures, shall be on site when fuel is in the reactor;
- d. ALL CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Operator or licensed Senior Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation;
- e. A site Fire Brigade of at least 5 members* shall be maintained onsite at all times. The Fire Brigade shall not include the Shift Supervisor, and the two other members of the minimum shift crew necessary for safe shutdown of the Unit and any personnel required for other essential functions during a fire emergency; and

*May be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence provided immediate action is taken to fill the required positions.

ADMINISTRATIVE CONTROLS

UNIT STAFF (Continued)

- f. Administrative procedures shall be developed and implemented to limit the working hours of Unit Staff who perform safety-related functions; e.g., Senior Operators, Operators, Health Physicists, Auxiliary operators, and key maintenance personnel.

The amount of overtime worked by Unit Staff members performing safety-related functions shall be limited in accordance with the NRC Policy Statement on working hours (Generic Letter No. 82-12).

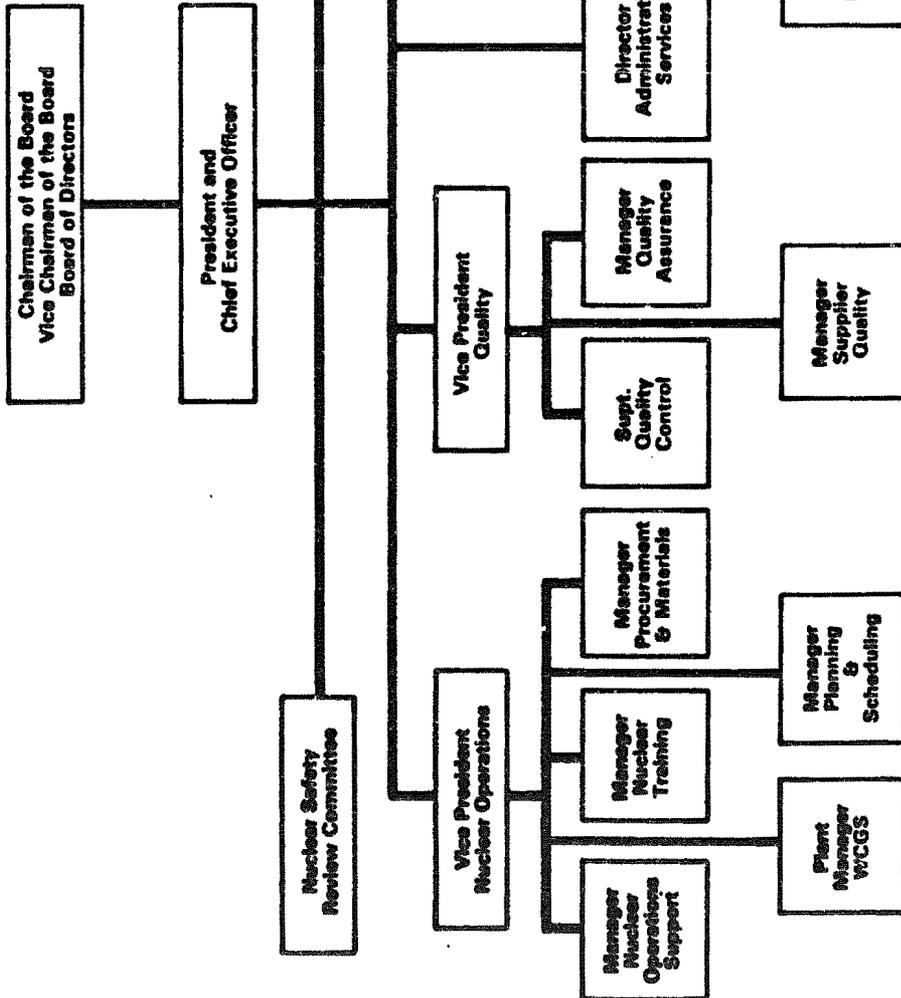
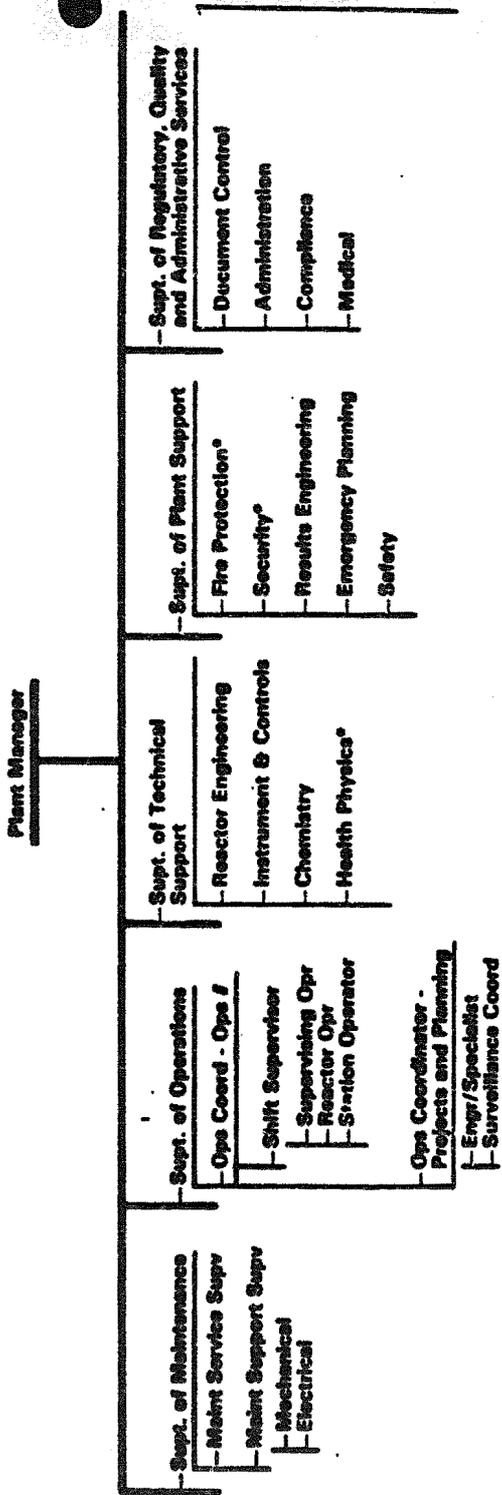


Figure 6.2-1 Operating Corporation Organization



* For technical matters of an immediate nature the respective individual report directly to the Plant Manager.

/ This position requires an SRD License.

FIGURE 6.2-2 UNIT ORGANIZATION

ADMINISTRATIVE CONTROLS

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Training Manager and shall meet or exceed the requirements and recommendations of Section 5 of ANSI/ANS 3.1-1978 and Appendix "A" of 10 CFR Part 55 and the supplemental requirements specified in Section A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience identified by the ISEG or another plant group.

6.5 REVIEW AND AUDIT

6.5.1 PLANT SAFETY REVIEW COMMITTEE (PSRC)

FUNCTION

6.5.1.1 The PSRC shall function to advise the Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PSRC shall be composed of the:

Chairman:	Plant Manager
Member:	Superintendent of Operations
Member:	Superintendent of Technical Support
Member:	Superintendent of Maintenance
Member:	Instrument and Control Supervisor
Member:	Reactor Engineering Supervisor
Member:	Health Physicist
Member:	Chemist
Member:	Results Engineering Supervisor
Member:	Superintendent of Plant Support
Member:	Superintendent of Regulatory, Quality and Administrative Services

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PSRC Chairman to serve on a temporary basis; however no more than two alternates shall participate as voting members in PSRC activities at any one time.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The PSRC shall meet at least once per calendar month and as convened by the PSRC Chairman or his designated alternate.

QUORUM

6.5.1.5 The quorum of the PSRC necessary for the performance of the PSRC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The PSRC shall be responsible for:

- a. Review of: (1) all procedures required by Specification 6.8 and changes thereto, (2) all programs required by Specification 6.8 and changes thereto, and (3) any other proposed procedures or changes thereto as determined by the Plant Manager to affect nuclear safety;
- b. Review of all proposed changes, tests and experiments which may involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- c. Review of all proposed changes to Technical Specifications or the Operating License;
- d. Review of all safety evaluations performed under the provision of Section 50.59(a)(1), 10 CFR, for changes, tests and experiments;
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President, Nuclear Operations, and to the Nuclear Safety Review Committee (NSRC);
- f. Review of all REPORTABLE EVENTS;
- g. Review of reports of operating abnormalities, deviations from expected performance of plant equipment and of unanticipated deficiencies in the design or operation of structures, systems or components that affect nuclear safety;
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman, NSRC;
- i. Review of the plant Security Plan and implementing procedures and shall submit recommended changes to the NSRC;
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the NSRC;

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- k. Review of changes to the PROCESS CONTROL PROGRAM, the OFFSITE DOSE CALCULATION MANUAL and the Radwaste Treatment Systems, and
 - l. Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Plant Manager and to the Nuclear Safety Review Committee.
- 6.5.1.7 The PSRC shall:
- a. Recommend in writing to the Plant Manager approval or disapproval of items considered under Specification 6.5.1.6a. through d. above,
 - b. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.1.6a. through e. above constitutes an unreviewed safety question, and
 - c. Provide written notification within 24 hours to the Vice President Nuclear Operations and the Nuclear Safety Review Committee of disagreement between the PSRC and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1 above.

RECORDS

6.5.1.8 The PSRC shall maintain written minutes of each PSRC meeting that, at a minimum, document the results of all PSRC activities performed under the responsibility provisions of these Technical Specifications. Copies shall be provided to the Vice President Nuclear Operations and the Nuclear Safety Review Committee.

6.5.2 NUCLEAR SAFETY REVIEW COMMITTEE (NSRC)

FUNCTION

6.5.2.1 The NSRC shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,

ADMINISTRATIVE CONTROLS

FUNCTION (Continued)

- g. Mechanical and electrical engineering, and
- h. Quality assurance practices.

The NSRC shall report to and advise the President and Chief Executive Officer on those areas of responsibility specified in Specifications 6.5.2.7 and 6.5.2.8.

COMPOSITION

6.5.2.2 The NSRC shall be composed of at least the following:

Chairman:	Manager Nuclear Services
Member:	Vice President Engineering and Technical Services
Member:	Manager Quality Assurance
Member:	Vice President Nuclear Operations
Member:	Manager Licensing
Member:	KCP&L Senior Director of Nuclear Affairs
Member:	Manager Nuclear Safety
Member:	Nuclear Coordinator (KG&E)

Additional members and Vice Chairman may be appointed by the Chairman.

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the NSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in NSRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the NSRC Chairman to provide expert advice to the NSRC.

MEETING FREQUENCY

6.5.2.5 The NSRC shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per 6 months thereafter.

QUORUM

6.5.2.6 The quorum of the NSRC necessary for the performance of the NSRC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least two-thirds of the NSRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the Unit.

ADMINISTRATIVE CONTROLS

REVIEW

6.5.2.7 The NSRC shall be responsible for the review of:

- a. The safety evaluations for: (1) changes to procedures, equipment, systems or facilities, and (2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question;
- b. Proposed changes to procedures, equipment, systems, or facilities which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- d. Proposed changes to Technical Specifications or this Operating License;
- e. Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety;
- g. All REPORTABLE EVENTS;
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety; and
- i. Reports and meeting minutes of the PSRC.

AUDITS

6.5.2.8 Audits of Unit activities shall be performed under the cognizance of the NSRC. These audits shall encompass:

- a. The conformance of Unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months;
- b. The performance, training and qualifications of the entire Unit Staff at least once per 12 months;
- c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems, or method of operation that affect nuclear safety at least once per 6 months;

ADMINISTRATIVE CONTROLS

AUDITS (Continued)

- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50, at least once per 24 months;
- e. The fire protection programmatic controls including the implementing procedures at least once per 24 months by qualified licensee QA or ISEG personnel;
- f. The fire protection equipment and program implementation at least once per 12 months utilizing either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used at least every third year;
- g. The Radiological Environmental Monitoring Program and the results thereof at least once per 12 months;
- h. The ODCM and implementing procedures at least once per 24 months;
- i. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months;
- j. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring at least once per 12 months;
- k. The Emergency Plan and implementing procedures at least once per 12 months;
- l. The Security Plan and implementing procedures at least once per 12 months; and
- m. Any other area of Unit operation considered appropriate by the NSRC or the President and Chief Executive Officer.

RECORDS

6.5.2.9 Records of NSRC activities shall be prepared, approved, and distributed as indicated below:

- a. Minutes of each NSRC meeting shall be prepared, reviewed by participating members and forwarded to the President and Chief Executive Officer within 14 days following each meeting;
- b. Reports of reviews encompassed by Specification 6.5.2.7 above, shall be prepared, reviewed by participating members and forwarded to the President and Chief Executive Officer within 14 days following completion of the review; and

ADMINISTRATIVE CONTROLS

RECORDS (Continued)

- c. Audit reports encompassed by Specification 6.5.2.8 above, shall be forwarded to the President and Chief Executive Officer and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 of 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the PSRC and submitted to the NSRC and the President and Chief Executive Officer.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The President and Chief Executive Officer and the NSRC shall be notified within 24 hours;
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PSRC. This report shall describe: (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective ACTION taken to prevent recurrence;
- c. The Safety Limit Violation Report shall be submitted to the Commission, the NSRC and the President and Chief Executive Officer within 14 days of the violation; and
- d. Critical operation of the Unit shall not be resumed until authorized by the Commission.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A, of Regulatory Guide 1.33, Revision 2, February 1978;
- b. The emergency operating procedures required to implement the requirements of NUREG-0737 and Supplement 1 to NUREG-0737 as stated in Section 7.1 of Generic Letter No. 82-33;
- c. Security Plan implementation;
- d. Emergency Plan implementation;

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PROCEDURES AND PROGRAMS (Continued)

- e. Process Control Program implementation;
- f. ODCM implementation; and
- g. Quality Assurance Program implementation for effluent and environmental monitoring.

Major Procedures, supported by appropriate Minor Procedures (such as checkoff lists, operating instructions, data sheets, alarm responses, etc.), shall be provided for the above activities.* A Major Procedure is a procedure which controls safety-related activities, and establishes one or more basic controls, overall responsibilities, authority assignments or administrative and operational ground rules at the Wolf Creek plant. Major Procedures are written to meet the requirements of ANSI N18.7-1976/ANS 3.2 and generally are supported by Minor Procedures which provide delineation of details such as for valve lineups, calibration procedures, operating instructions, data sheets, alarm responses, and other procedures identified as "supporting." Major Procedures require signature approval in all cases by the Plant Manager or a Call Superintendent in his absence. A Minor Procedure is a procedure which controls safety-related activities in support of a Major Procedure. It addresses a specific topic or sub-topic established by its 'parent' Major Procedure, expanding on it by providing working level instructions. Minor Procedures are not permitted to contradict requirements contained in their governing Major Procedure. Minor Procedures require signature approval by the Plant Manager, or a Call Superintendent in his absence, only at Revision '0.'

6.8.2 Approval of Procedures

- a. All Major Procedures of the categories listed in Specification 6.8.1 and modifications to the intent thereof shall be reviewed by the PSRC and approved by the Plant Manager prior to implementation and reviewed periodically as set forth in Administrative Procedures.
- b. Minor Procedures (checkoff lists, operating instructions, data sheets, alarm responses, chemistry and analytical procedures, technical instructions, special and routine maintenance procedures, laboratory manuals, etc.) shall, prior to initial use, be approved by the PSRC or a Subcommittee thereof.
- c. Corporate Emergency Plan implementing procedures shall be reviewed by appropriate corporate and plant personnel and approved by the President and Chief Executive Officer as set forth in General Procedures.

*With the exception of Corporate Emergency Plan implementing procedures. Corporate Emergency Plan implementing procedures shall be provided but shall not be designated as major or minor procedures.

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PROCEDURES AND PROGRAMS (Continued)

6.8.3 Changes to Procedures

- a. Temporary changes to Major Procedures, of the categories listed in Specification 6.8.1 which do not change the intent or generate an unreviewed safety question of the original or subsequent approved procedure, may be made provided such changes to operating procedures are approved by the Shift Supervisor (SRO licensed) and one of the Call Superintendents. For temporary changes to Major Procedures under the jurisdiction of Maintenance, Instrumentation and Control, Reactor Engineering, Chemistry, or Health Physics which do not change the intent or generate an unreviewed safety question, changes may be made upon approval of the Cognizant Group Leader and a Call Superintendent.

All temporary changes to Major Procedures (made by a Call Superintendent and either a Cognizant Group Leader or the Shift Supervisor) shall subsequently be reviewed by the PSRC and approved by the Plant Manager within 14 days, except that temporary changes to Major Procedures made during a refueling outage may be reviewed and approved at any time prior to initial criticality of the reload core. All permanent changes to Major Procedures shall be made in accordance with Specification 6.8.2.a.

- b. All temporary or permanent changes to Minor Operating Procedures (checkoff lists, alarm responses, data sheets, operating instructions, etc.) shall be approved by the Shift Supervisor, and shall be subsequently reviewed and approved by the Operations PSRC Subcommittee. All temporary or permanent changes to other Minor Procedures under the jurisdiction of Maintenance, Instrumentation and Control, Reactor Engineering, Chemistry, or Health Physics, shall be approved by a Cognizant Group Leader and shall be subsequently reviewed and approved by the appropriate PSRC Subcommittee.
- c. Temporary changes to Corporate Emergency Plan implementing procedures may be made provided that: (1) the intent of the original procedure is not altered, (2) the change is approved by the Emergency Planning Coordinator, and (3) the change is documented, reviewed by appropriate Corporate and plant personnel and approved by the President and Chief Executive Officer within 14 days of the implementation.

6.8.4 The following programs shall be established, implemented, and maintained:

a. Reactor Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the appropriate portions of the Containment Spray System, Safety Injection System, Chemical and Volume Control System, RHR System, and the Nuclear Sampling System (PASS only). The program shall include the following:

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

- 1) Preventive maintenance and periodic visual inspection requirements, and
- 2) Integrated leak test requirements for each system at refueling cycle intervals or less.

b. In-Plant Radiation Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- 1) Training of personnel,
- 2) Procedures for monitoring, and
- 3) Provisions for maintenance of sampling and analysis equipment.

c. Secondary Water Chemistry

A program for monitoring of secondary water chemistry to inhibit steam generator tube degradation. This program shall include:

- 1) Identification of a sampling schedule for the critical variables and control points for these variables,
- 2) Identification of the procedures used to measure the values of the critical variables,
- 3) Identification of process sampling points, which shall include monitoring the discharge of the condensate pumps for evidence of condenser in-leakage,
- 4) Procedures for the recording and management of data,
- 5) Procedures defining corrective action for all off-control point chemistry conditions, and
- 6) A procedure identifying: (a) the authority responsible for the interpretation of the data, and (b) the sequence and timing of administrative events required to initiate corrective action.

d. Post-accident Sampling

A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

- 1) Training of personnel,
- 2) Procedures for sampling and analysis, and
- 3) Provisions for maintenance of sampling and analysis equipment.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 4 TO OPERATING LICENSE NO. NPF-42

KANSAS GAS & ELECTRIC COMPANY

KANSAS CITY POWER AND LIGHT COMPANY

KANSAS ELECTRIC POWER COOPERATIVE, INC.

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

INTRODUCTION

By letter dated April 15, 1986, as supplemented July 29, 1986, Kansas Gas and Electric Company (KG&E), on behalf of the Wolf Creek owners, requested that the Wolf Creek Operating License and Appendix A (Technical Specifications) be amended to allow licensed activities to be under the control of the Wolf Creek Operating Corporation, a new corporation jointly established by the Wolf Creek owners. This transfer of operating responsibility from KG&E to the Wolf Creek Operating Corporation would become effective on January 1, 1987.

Currently, Kansas Gas and Electric Company (KG&E), Kansas City Power & Light Company (KCPL) and Kansas Electric Power Cooperative, Inc. (KEPCO) are the holders of Facility Operating License No. NPF-42, dated June 4, 1985. The operating license authorizes KG&E, KCPL and KEPCO to possess the Wolf Creek Generating Station, Unit 1 (WCGS) and authorizes KG&E as agent for the Owners, to use and operate the plant in accordance with the procedures and limitations set forth in the operating license.

The owners have jointly established a new corporation, the Wolf Creek Nuclear Operating Corporation, to assume all responsibilities for operating WCGS now held by KG&E under the operating license. Ownership of WCGS will remain with the owners and will not be transferred to the Operating Corporation. The stock of the Operating Corporation is owned by KG&E, KCPL and KEPCO in proportion to their respective ownership shares of WCGS i.e. KG&E (47%), KCPL (47%) and KEPCO (6%). Five members of the board of Directors are designated by KG&E, five members by KCPL and two by KEPCO. The thirteenth director will be the, as yet undesignated, President and Chief Executive Officer (CEO) of the Operating Corporation (to be elected by the remaining members of the Board of Directors). The members of the Board of Directors will elect one of their members as Chairman of the Board to preside at meetings of the Board.

EVALUATION

The technical qualifications of the Operating Corporation to carry out its responsibilities under the operating license will be the same as the technical

qualifications of KG&E. All KG&E Nuclear Department employees, all other KG&E employees assigned full-time to WCGS matters, and such KCPL and KEPCO employees who are assigned full-time to WCGS will automatically become employed by the Operating Corporation. Also, all KG&E employees who perform services related to the KG&E Nuclear Department on a part-time basis and who are requested by the Operating Corporation for transfer to it and agreed to by KG&E will automatically become employed by the Operating Corporation.

The same KG&E organization and staff which is currently responsible for operating WCGS will continue those responsibilities as part of the Operating Corporation. The changes between the current and proposed structure are: (1) the Directors of Nuclear Operations, Engineering and Technical Services, and Quality, become Vice Presidents of the Operating Corporation; (2) a General Counsel and Secretary have been added to the Operating Corporation organization; (3) the Construction Division has been merged into the Engineering and Technical Services Branch; and (4) some other minor title changes are reflected. The Operating Corporation's Vice Presidents, the Director Administrative Services and the Nuclear Safety Review Committee, with the newly created General Counsel and Secretary, will report to the President and Chief Executive Officer of the Operating Corporation rather than to KG&E's Vice President Nuclear. The President and Chief Executive Officer of the Operating Corporation will report directly to the Board of Directors of the Operating Corporation; presently KG&E's Vice President Nuclear reports to KG&E's Group Vice President Technical Services who in turn reports to KG&E's President and Chairman of the Board.

The licensee states that there will be no change in the numbers and qualifications of personnel who operate WCGS as a result of the license amendment. Therefore, we conclude that implementation of the amendment as proposed will cause no change in the current technical qualifications. The Technical Specifications affected are 6.1.2, 6.2.1, 6.5.1, 6.5.2, 6.6.1, 6.7.1, 6.8.2, and 6.8.3. These are title changes and do not affect current reporting relationships or personnel qualifications. The licensee has stated that there will be no change in qualifications of the personnel who operate WCGS as a result of the license amendment. The proposed change will involve no alterations to the facility itself and no substantive modifications to plant procedures. The NRC staff has determined that the proposed Operating Corporation is an entity that generates electricity and recovers the costs of that electricity indirectly through rates established by separate regulatory authorities. Accordingly, the Operating Corporation is an "electric utility" as defined in 10 CFR 50.2 and is therefore not subject to the requirements of 10 CFR 50.33(f) related to the applicants financial qualification.

Based on our review of the proposed license amendment, the staff finds the proposed amendment to Operating License No. NPF-42 and Appendix A to the license acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in recordkeeping, reporting or administrative procedures or requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (51 FR 29002) on August 13, 1986, and consulted with the state of Kansas. No public comments were received, and the state of Kansas did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

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Dated: November 4, 1986